

## **Experiments at JRC-Geel using MINERVE samples**

**G. Noguere, P. Leconte, M. Pottier, S. Martin, CEA, DES, IRESNE Cadarache, France**

**C. Paradela, S. Kopecky, P. Schillebeeckx, European Commission, Joint Research Centre Geel, Belgium.**

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## Subtask 5.3.1 Experiments at GELINA

The proposed experiments will consist in performing neutron transmission measurements at the JRC Geel GELINA facility using the same samples as those used in the CEA Cadarache MINERVE reactor as part of the past Burnup Credit program.

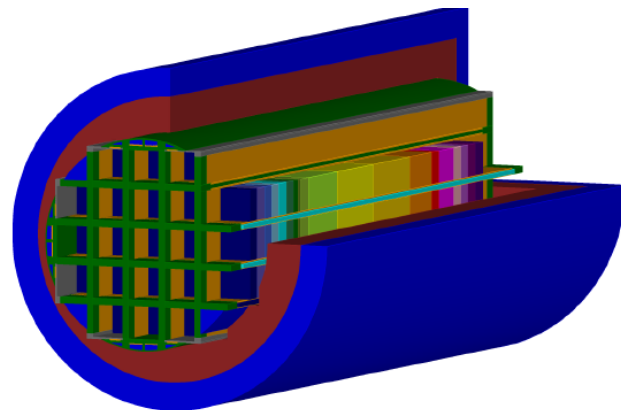
Preliminary studies show that such experiments should be feasible. Each sample is made of a UO<sub>2</sub> matrix with a small admixture of a fission product: Sm, Nd, Cs, Mo, Ru, Eu, Gd, or Rh. The expected outcome will be a set of transmission data for each sample. These data will be first used to determine the amount of contaminants in the samples by Neutron Resonance Transmission Analysis (NRTA).

# Burnup Credit definition

**Burnup-Credit (BUC)** : safety approach that accounts for the reduction of the spent fuel reactivity due to its burnup (reduction of net fissile content, actinides build-up, increase of **fission products** concentration)

**Applications:** spent fuel storage, transport cask

⇒ In this talk, we are interested by the contribution of the fission products

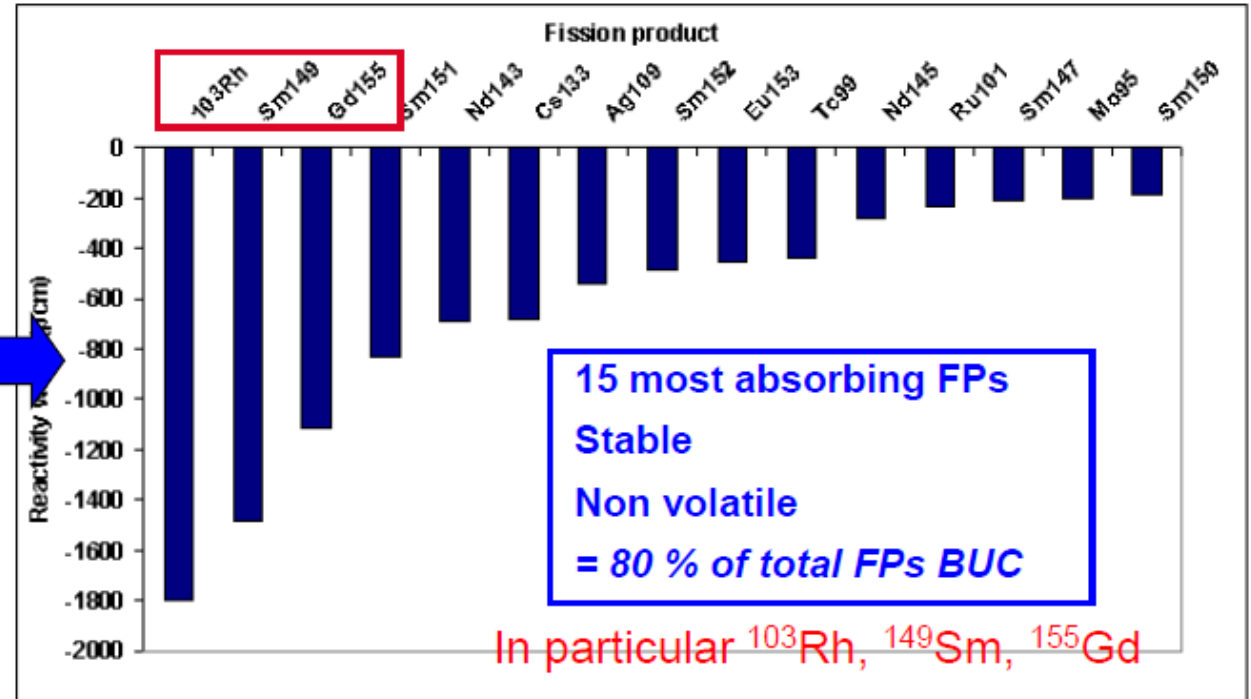


# Fission product contribution

Fission product contribution in BUC calculations for UOX and MOX fuels (end of cycle)

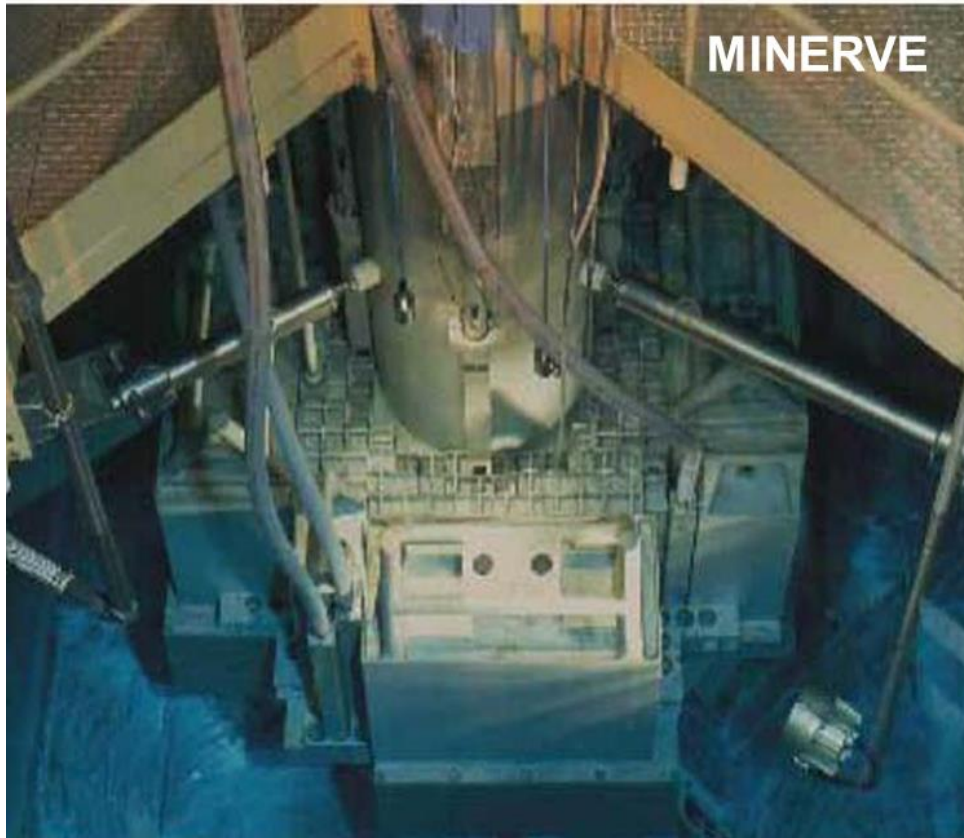
	PWR-UOx	PWR-MOX
Actinide BUC	19000 pcm	7550 pcm
15 FPs BUC	8400 pcm	8330 pcm
<b>Total BUC</b>	<b>27400 pcm</b>	<b>15880 pcm</b>

BU = 40 GWd/t<sub>HM</sub> Cooling time 1 year



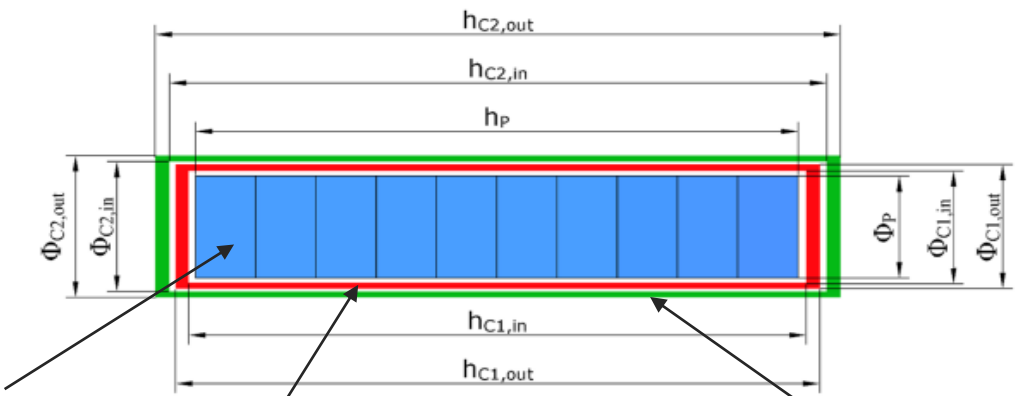
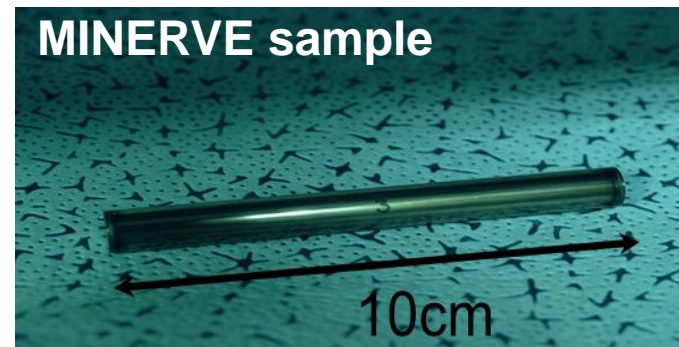
⇒ Calculation biases in BUC applications relies on integral measurements, such as reactivity worth measurements carried out in the **MINERVE reactor of CEA Cadarache**

# MINERVE measurements for BUC applications



Reactivity worth measurements in the MINERVE reactor of CEA Cadarache using the so-called **MINERVE samples**

**MINERVE samples**  
cylindrical samples in which fission product are mixed to  $\text{UO}_2$ ,  $\text{Al}_2\text{O}_3$  or liquid.

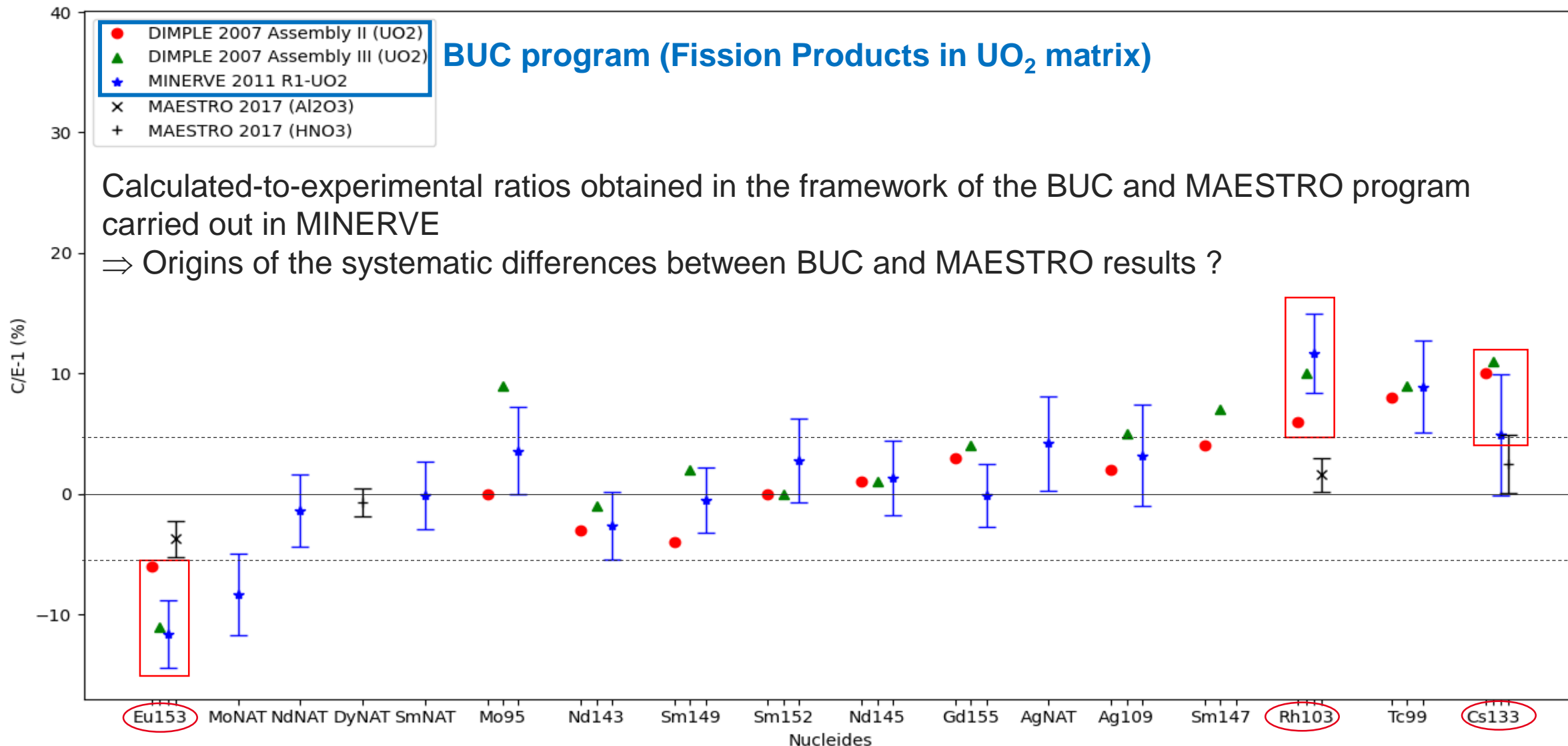


$\text{UO}_2$  pellet containing fission product

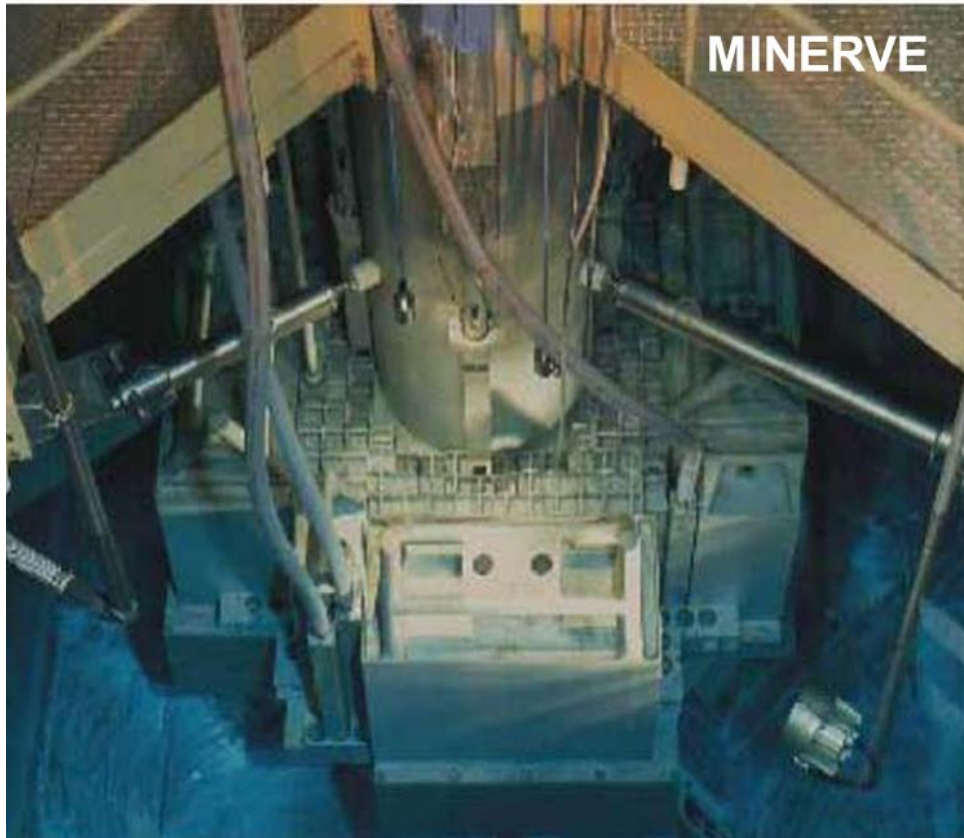
Zircaloy-4 cladding

Zircaloy-4 cladding

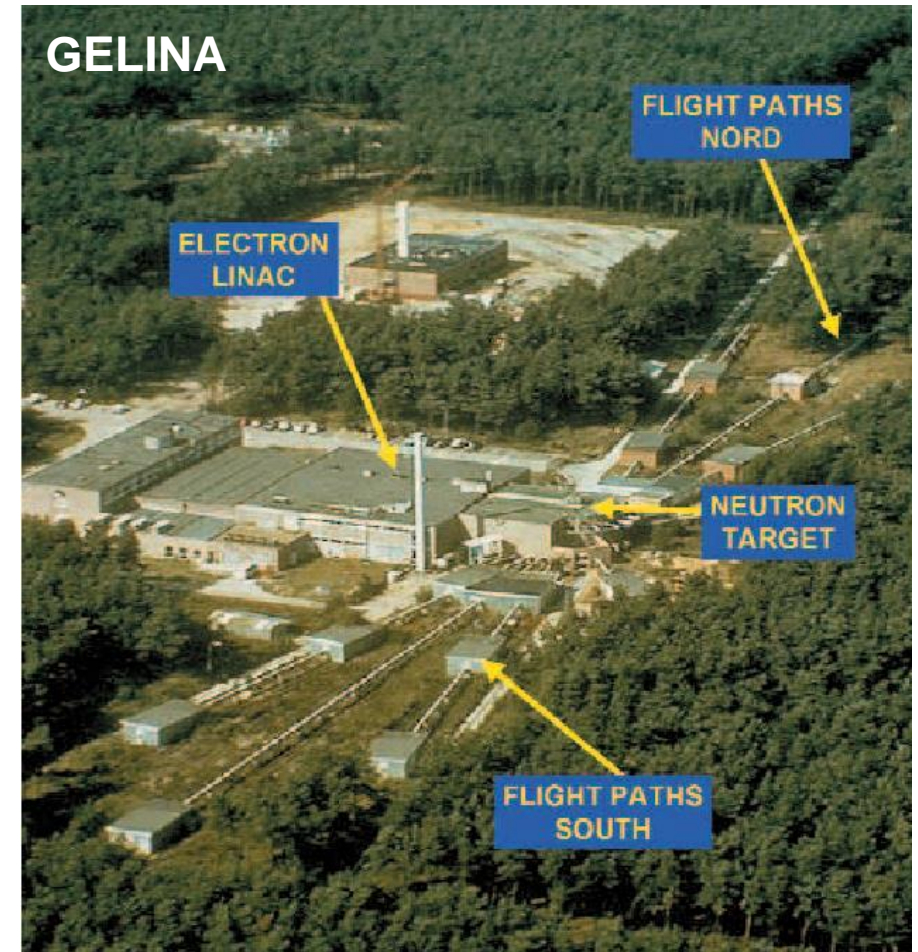
# MINERVE results with JEFF-3.1.1



# GELINA measurements for BUC applications



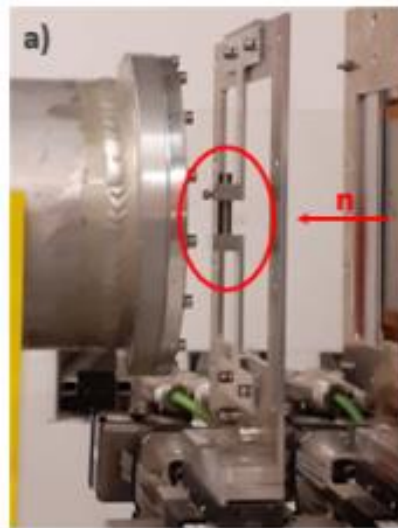
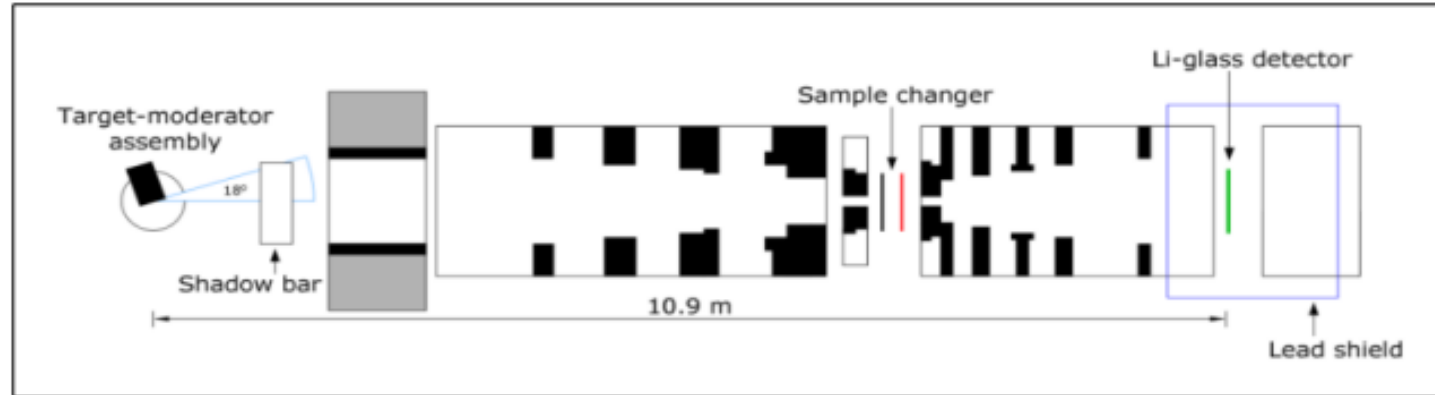
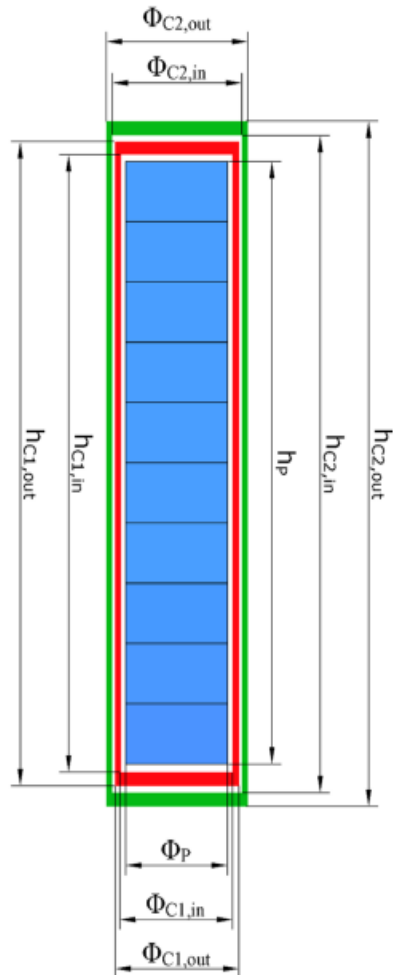
Reactivity worth measurements in the MINERVE reactor of CEA Cadarache using the so-called **MINERVE samples**



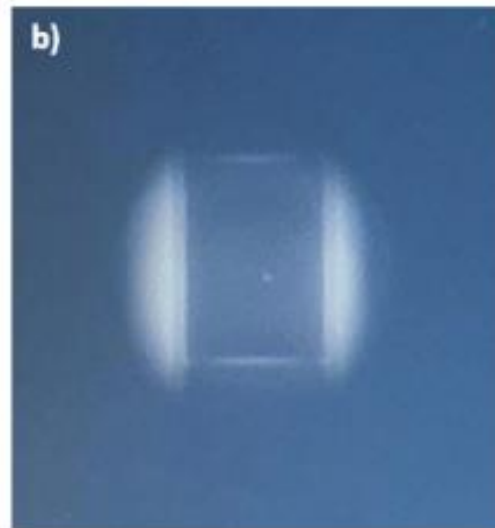
**MINERVE samples** can also be measured at the GELINA facility (JRC-Geel, Belgium) to verify integral results obtained with the MINERVE reactor

# Principles of transmission measurement

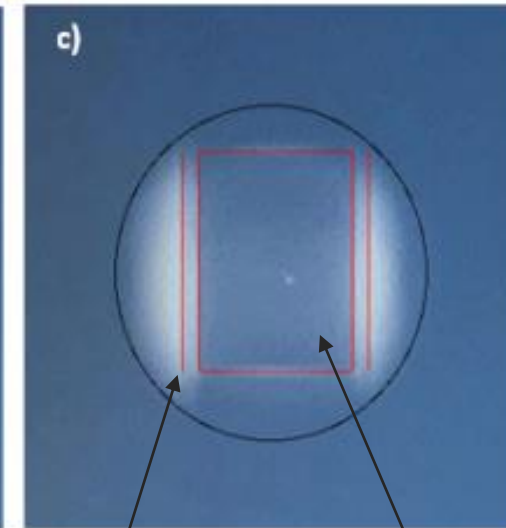
Transmission measurements were performed at L=10.9 m from the neutron source (FP13)



**MINERVE sample**  
placed vertically in  
the neutron beam



Picture of the  
MINERVE  
sample in the beam

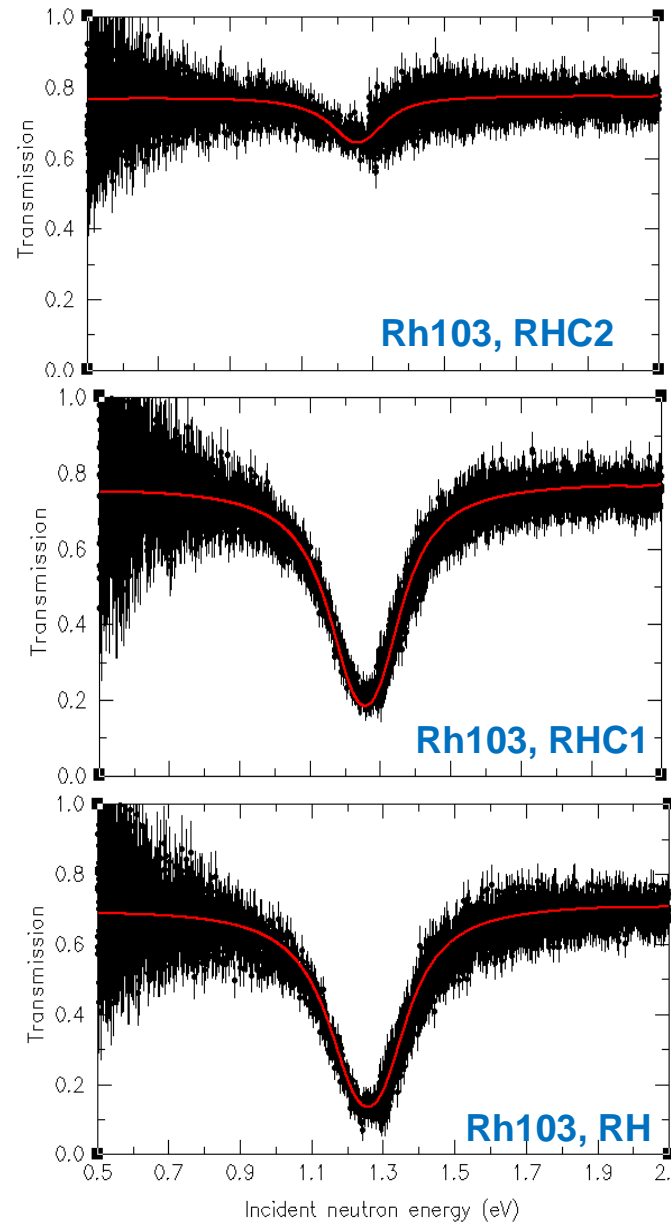
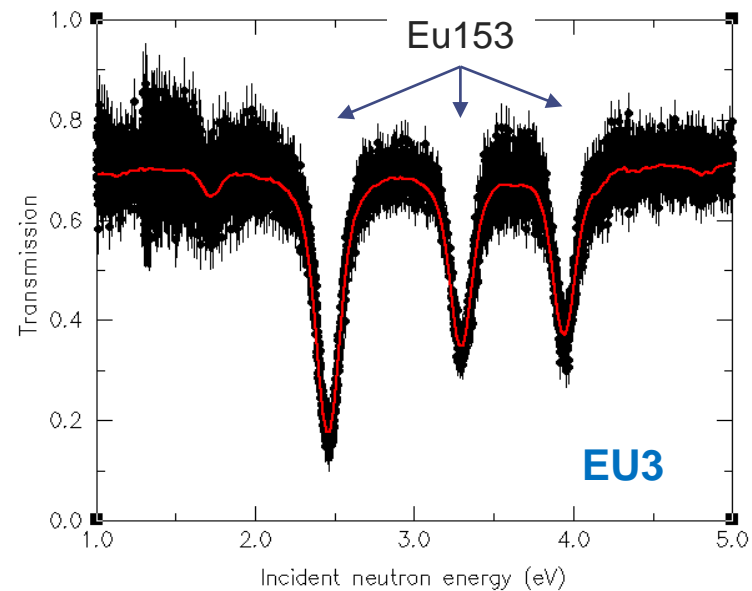
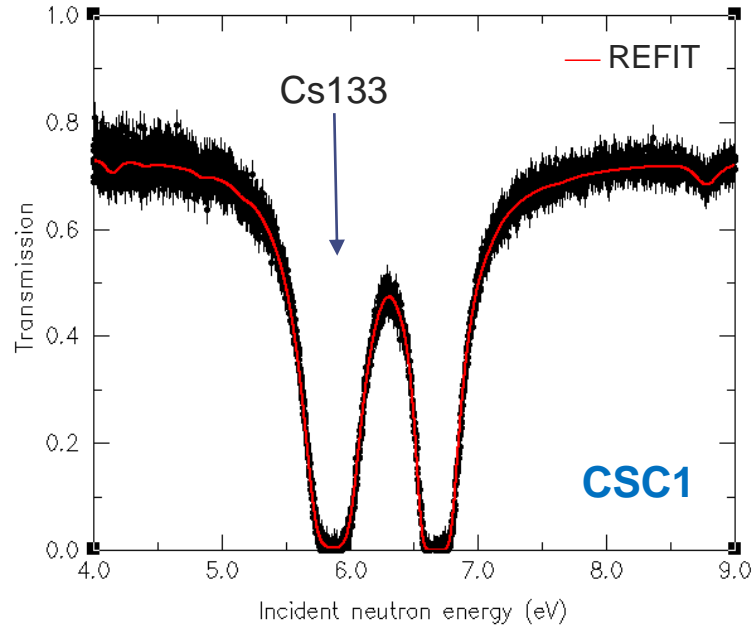


Zircaloy-4  
cladding

UO<sub>2</sub> pellet  
containing  
fission product



# Results (1/4)

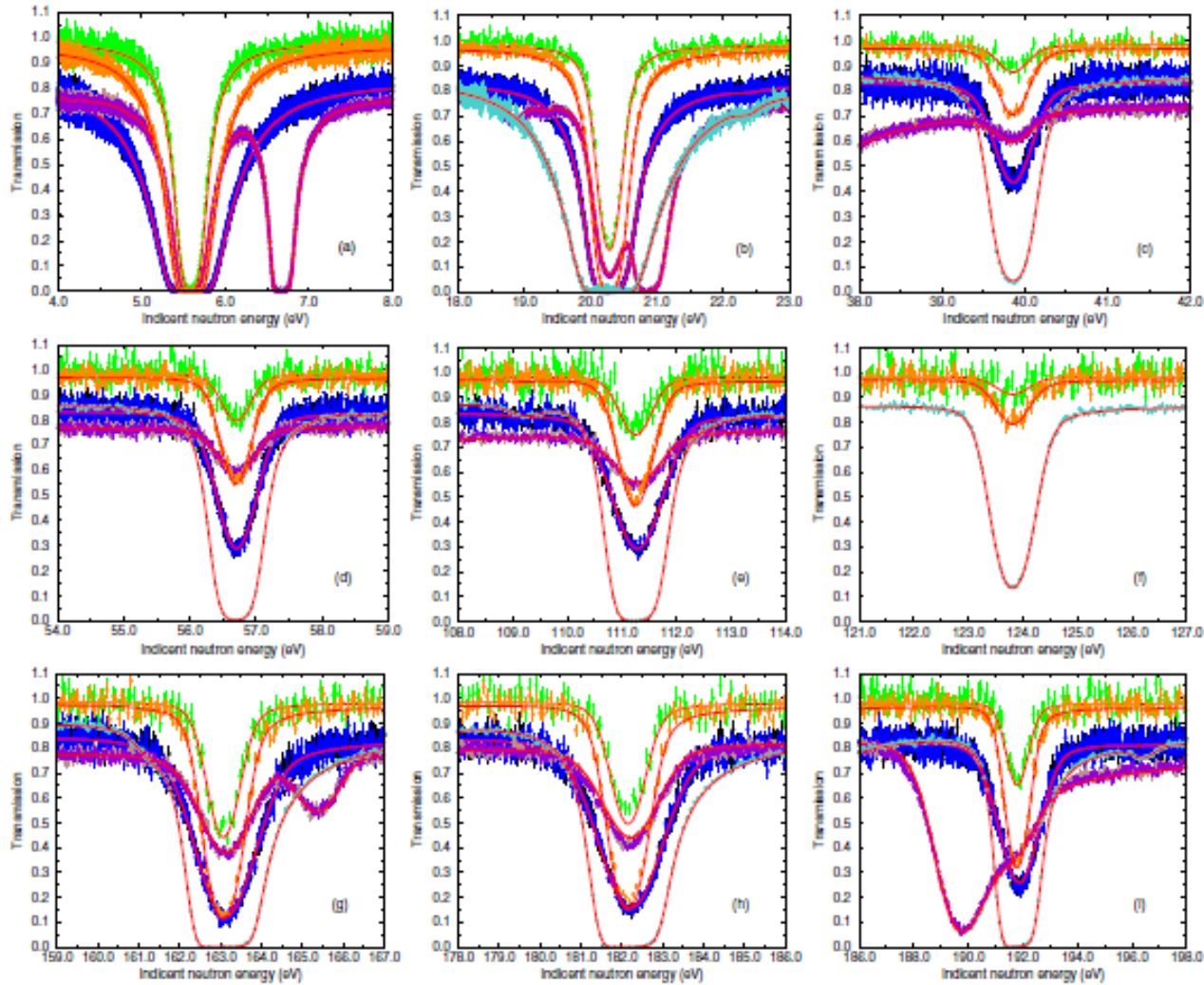


Good agreement between theoretical transmission (in red) and data (in black)



GELINA confirmed  
MINERVE sample composition  
declared by the manufacturer

# Results (2/4)



Tc99

Simultaneous analysis of various transmission data measured at the GELINA facility



GELINA contributes to improve nuclear data for the JEFF library

Joint Evaluated Fission and Fusion (JEFF) Nuclear Data Library

JEFF-4T3 (Official) Downloads

Welcome to the official release of the JEFF-4T3 library!

After our initial unofficial release in September 2023, we are thrilled to announce that JEFF-4T3 is now officially available as of February 2024. This release marks a significant milestone in our continuous efforts to provide accurate and reliable nuclear data to the scientific community.

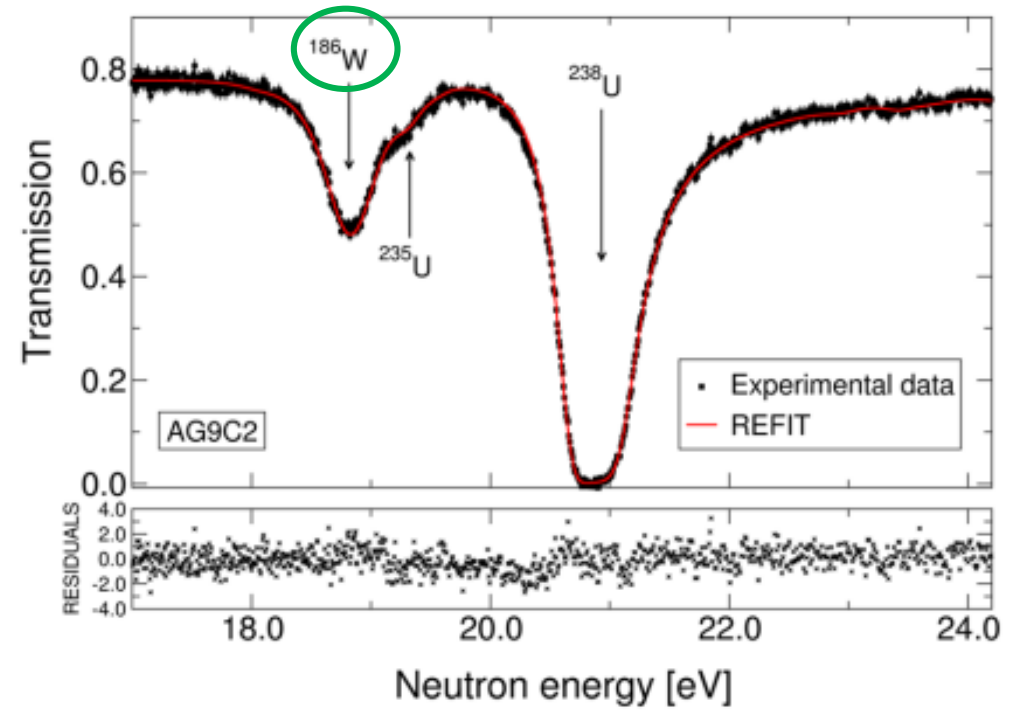
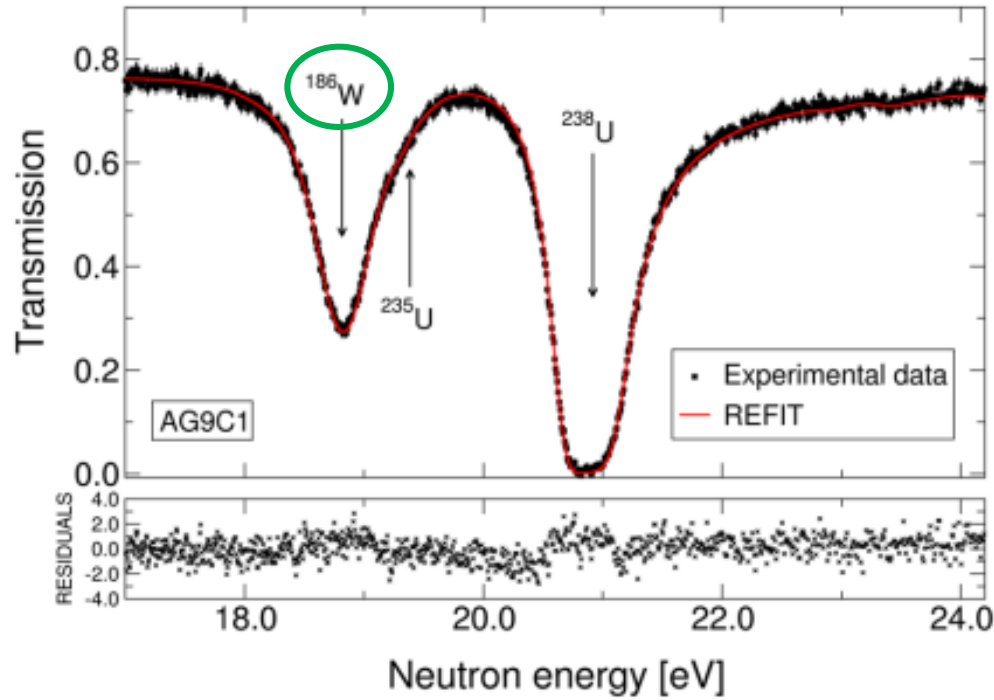
For a comprehensive overview of the changes introduced in JEFF-4T3, we invite you to refer to this spreadsheet: JEFF-4 Evolution. It provides detailed insights into the modifications made.

**Kindly be aware that the most recent official release from JEFF continues to be JEFF-3.3.**

**Fission Yields (FY):**

Release	Format	Content	Files	Download
JEFF-4T3	ENDF	Thermal fission of U-235 (Conservative approach)	4	↓
JEFF-4T3	ENDF	Thermal fission of Pu-239 (Conservative approach)	4	↓

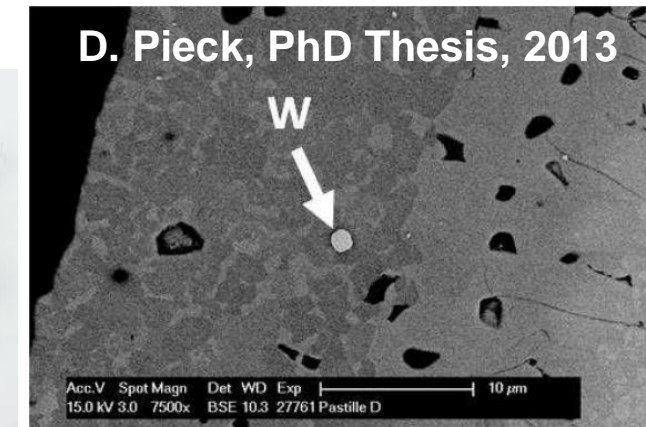
# Results (3/4)



GELINA highlighted a tungstene contamination not reported by the manufacturer of the MINERVE samples



Erosion of tungsten carbide balls used for powder grinding



## Results (4/4)

⇒ Volume number density of the tungsten isotopes in the MINERVE samples provided by GELINA

Sample	$^{182}\text{W}$ (26.50 at%)	$^{183}\text{W}$ (14.31 at%)	$^{184}\text{W}$ (30.64 at%)	$^{186}\text{W}$ (28.43 at%)
AG9C1	$1.59 \times 10^{-5}$	$8.56 \times 10^{-6}$	$1.83 \times 10^{-5}$	$1.70 \times 10^{-5}$
AG9C2	$7.30 \times 10^{-6}$	$3.94 \times 10^{-6}$	$8.43 \times 10^{-6}$	$7.82 \times 10^{-6}$
UNC	$2.06 \times 10^{-6}$	$1.11 \times 10^{-6}$	$2.37 \times 10^{-6}$	$2.20 \times 10^{-6}$
AAG	$1.63 \times 10^{-6}$	$8.80 \times 10^{-7}$	$1.89 \times 10^{-6}$	$1.75 \times 10^{-6}$

The densities are expressed in  $10^{24}$  at/cm<sup>3</sup>

⇒ Impact of tungsten on the calculated reactivity worth

MINERVE sample	Impact W on the calculated reactivity worth
AG9C1	+1.5 %
AG9C2	<b>+9,9 %</b>

**Large impact of the tungsten contamination**



GELINA suggests that the reactivity worth measured with the MINERVE sample AG9C2 is not trustable

# Conclusions

## 30 MINERVE samples containing fission products available at JRC-Geel for transmission measurements

- Tc99, Ag109, Ag107, Rh103, Gd155, Eu153, Cs133, Gd<sub>2</sub>O<sub>3</sub> completed

## GELINA measurements confirm the composition provided by the manufacturer of the MINERVE samples ...

- Differences between volume densities declared by the manufacturer and GELINA results close to 2%

## ... and highlighted a non-negligible tungsten contamination

- New interpretation of the pile oscillation measurements with the GELINA results in progress